Form 2  

<table>
<thead>
<tr>
<th>Technology Information</th>
</tr>
</thead>
<tbody>
<tr>
<td><strong>Area</strong></td>
</tr>
<tr>
<td><strong>Title</strong></td>
</tr>
<tr>
<td><strong>Submitted by</strong></td>
</tr>
</tbody>
</table>

1. **Overview of Technologies** *(See Appendix for complete list and sorption information.)*

We have over 20 years of success in the development and specific tuning of novel materials to needed selectivity as sorbent and high durability as waste forms. Below is a brief review of the most relevant materials for the needs at Fukushima in the area of Removal of radioactive materials from the seawater in the harbor (3). See Figure 1.

With our expertise, we envision the use of our new materials in addition to our CSTs for the removal of radioactive Cesium and Strontium from the sea. As noted below, we have developed highly selective materials for Cs and Sr which are not selective to ions of concern; in particular the SOMS materials which do not have selectivity for Na, Mg, Ca, thereby dramatically decreasing the concern of interference with target ion sorption. The amount of waste will be limited as these inorganic sorbents are highly selective for the target ions, even in seawater. Since the CSTs have been successfully integrated in sorption columns, we anticipate our other materials (which are similar in chemistry) will behave similarly and be easily incorporated either into a similar system or as sorbents inside silt fences or nets. Furthermore, we describe our novel technology for the easy incorporation of the sorbents into a durable new glass composite material (GCM) waste form. Note, much of the data associated with the selectivity of these materials is in the Appendix.

- **CSTs:** We discovered and developed the Crystalline Silicotitanates (CSTs, Sandia owned and patented # 6,479,427 and # 6,110,378) in the mid 1990s for Cs$^+$ removal from mixed aqueous streams over the entire pH scale. (see Figure 2) UOP licensed and commercialized it (products names IONSIEV IE-910 and IE-911) for use in US waste sites. April 2011, I was contacted by the US Dept of Energy to analyze CSTs for Cs+ removal from seawater. It outperformed all manufactured and mineral zeolites and clays (published 2011). Honeywell UOP LLC relicensed the CSTs from Sandia in 2011 and restarted large-scale production for use in the SARRY process, currently in use at Fukushima. It is a pelletized ceramic molecular sieve and used in aqueous columns or batch systems; can be incorporated into mesh nets.

- **Silicotitanate Sorption Materials:** I co-discovered and developed a number of
silicotitanate sorption and in-situ sorption/waste form materials that are highly selective for Cs+ (Sandia owned and patented #6,482,380). They are ceramic based and used in aqueous columns or batch systems; can be incorporated into mesh nets.

- Waste Form: My team and I have proven that CSTs can successfully be incorporated into Glass waste forms (Borosilicate Glass; Sandia low temp sintering Bi-Si glass waste form (Sandia owned and patented #8,262,950)). It can also be heat treated and form a 1-step ceramic pollucite waste form stable to Cs+ -> Ba2+ oxidation state changes with decay.

- SOMS: I co-discovered and developed the Sandia Octahedral Molecular Sieves (SOMS, Sandia owned and patented #7,122,164 and # 6,596,254) for Sr2+ removal over the entire pH scale. It is highly selective for Sr2+ over monovalent cations and against competing (e.g., Na+, Cs+) cations. (See figures 3a-c) It is a molecular sieve easily pelletized and used in aqueous columns or batch systems; can be incorporated into mesh nets. These phases have high selectivity for various metal ions also (see patent). Gamma irradiation testing completed with no detriment to the sorption material or loss of the ion.

- Waste Form: I and my team have proven that Sr-SOMS can be heat treated at 450°C to perovskites (with no loss of Sr) to form a stable ceramic waste form or be incorporated into our durable Sandia low temp sintering Bi-Si glass waste form (Sandia owned and patented # #8,262,950).

- I co-discovered and developed an in-situ iodide and iodate immobilization process from aqueous streams; especially useful in caustic streams (Sandia owned and patented #8,383,021). This low energy, low cost process produces durable, mineral analog, Bi-O-I phases and can be performed in batch or stream processes, and importantly, in-situ to the aqueous stream. Gamma irradiation testing completed with no detriment to the resultant capture material / waste form.

- I co-discovered and developed (Sandia owned and US Patented # 8,262,950) a low temperature sintering, highly durable, glass based “universal” Waste Form that is Bi-Si-O based and that successfully incorporates known getter materials for nuclear industrial sorption; it forms a Glass Composite Material waste form (GCM). Durability studies of SPFT, PCT and MCC-1 indicate it has durability greater or equal to nuclear waste glasses. We have successfully included > 20wt % rad-gas or rad-ion loaded zeolites, clays, molecular sieves, metal-organic frameworks (MOFs), etc. The glass sinters at 550°C low enough that volatilization is negligible (e.g., AgI-Zeolite has no Iodine loss). Gamma irradiation testing completed with no detriment to the waste form. (See figures 4a-b)
- I co-discovered and developed (Sandia owned and US Patented 13/674,777 filed November 12, 2012, and international PCT patent application PCT/US2012/64698, filed November 12, 2012) a **binder-free pelletization** process that is successfully used on zeolites, molecular sieves and metal-organic frameworks. Surface area, sorption capacity, durability, crystallinity, etc. are all retained post pelletization process. (See figure 5)

- I co-discovered and developed **metal-organic frameworks (MOFs)** for the sorption of radiological fission gases (eg., iodine gas, I₂), and the 1-step formation of MOF-I₂ interim waste form via amorphization. (Sandia owned and US and PCT Patent Applications filed October 23, 2012.) This allows for high sorption capacity and non-HIPping easy mechanical pressing to collapse overall MOF framework and destroying pathway for gas escape, thereby rendering material an interim waste form. Our studies show enhanced gas retention with time and temperature. Gamma irradiation testing completed with no detriment to the sorption material or loss of the gas.

1. **Notes**
   - **Gamma irradiation** testing of materials (listed above either performed on radwaste (eg., CSTs in Fukushima and Milton Valley) or were irradiated at the Sandia Gamma Irradiation Facility (GIF; both (a) long-term exposure/low dose: 0.1 Rads/sec, with an overall dose of 2.59 x 10⁵ Rads (2218Gy), and (b) short-term exposure/high dose: 800 Rads/sec, 1 x 10⁶ Rads (10,000 Gy)

   - **Technology readiness level**
     - CSTs/IE-910 & IE-911: TRL 9 in use at Fukushima in SARRY process
     - Other Silicotitanates: TRL 3 (nuclear and metal ion removal)
     - SOMS: TRL 5 tested at Sandia, US waste sites, Russian sites (nuclear)
     - Bi-I-O: TRL 4 (nuclear)
     - Pollucite Cs/Si/Ti waste forms: TRL 5 (tested at SRNL; nuclear)
     - Low Temp Sintering Bi-Si glass waste forms: TRL 6 (durability testing completed; nuclear)
     - Pelletization Method: TRL 4 (nuclear and catalysis)

2. **Challenges:** Minimal Challenges, generally around scale up production. However ongoing collaborations in the production of CSTs enables a clear path forward with our materials.
   - The CSTs are commercially available, in production, and in use at Fukushima.
- The SOMS materials are similar in synthesis procedure; it needs scale-up and production.
- The basis of the low temperature Bi-Si Glass is a commercial product from Ferro Corporation (USA). The durability studies and tests are completed. Optimization of loading levels of sorption materials and scale up are needed.
- Bi-I-O needs to be tested on large scale.
- Pelletization needs to be tested on large scale.

3. **Others:**

    **Related Sandia & Nenoff patents:**
    3. Silico-titanates and their methods of making and using, 6,479,427, November 12, 2002
    7. Low Sintering Temperature Glass Waste Form for Sequestering Radioactive Iodine, 8,262,950, September 11, 2012

**Recent Publications that support the Patents:**


Areas of Technologies Requested

(1) Accumulation of contaminated water (Storage Tanks, etc.)
(2) Treatment of contaminated water (Tritium, etc.)
(3) **Removal of radioactive materials from the seawater in the harbor**
(4) Management of contaminated water inside the buildings
(5) Management measures to block groundwater from flowing into the site
(6) Understanding the groundwater flow