(修士課程)

・原子炉廃止措置工学

Engineering for nuclear decommissioning

(博士課程)

・原子核システム安全工学特論

Advanced safety engineering of nuclear systems

(公開講座)

・軽水炉安全セミナー(原子炉廃止措置編)

福島第一の廃炉研究開発の現状と課題

Current Status and Challenges of R&D for Decommissioning of Fukushima Daiichi Nuclear Power Station

September 26, 2023

技術研究組合 国際廃炉研究開発機構

業務管理部 部長

奥住 直明

Senior Manager Planning and Administration Department

International Research Institute for Nuclear Decommissioning (IRID)

Naoaki Okuzumi

The results are obtained under the Subsidy Project of Decommissioning and Contaminated Water Management granted by

the Ministry of Economy, Trade and Industry. Unauthorized copy and reproduction prohibited. ©International Research Institute for Nuclear Decommissioning

Contents

- 1. Introduction
- 2. Development of Remotely Operation Equipment for Dose Reduction
- 3. Development of Repairing Technology for PCV
- Development of Investigation Technology for inside PCV
- 5. Development of Technology for Fuel Debris Retrieval
- 6. Nuclear Safety Enhancement

PCV: primary containment vessel



Contents

1. Introduction

- 2. Development of Remotely Operation Equipment for Dose Reduction
- 3. Development of Repairing Technology for PCV
- 4. Development of Investigation Technology for inside PCV
- 5. Development of Technology for Fuel Debris Retrieval
- 6. Nuclear Safety Enhancement

PCV : Primary containment vessel



Video: Introduction of IRID



Boiling Water Reactor (BWR)



Appearance of Primary Containment Vessel (PCV) (Photo of PCV under construction)

Dry well (D/W): Upper part of PCV above the S/C

PCV penetration : Penetrations of piping and electric wiring, etc. Unit 1: Approx. 150 penetrations Unit 2: Approx. 200 penetrations Unit 3: Approx. 190 penetrations

Equipment hatch: Carry-in/out port of large equipment

Vent pipe: Connection pipe between D/W and S/C

Suppression Chamber (S/C): The S/C condenses water vapors generated when an accident occurs to suppress the increase of pressure in PCV.

"Browns Ferry Unit 1 under construction 1966.Sep." Tennessee Valley Authority – TVA's 75th Anniversary webpa



for humans

Air lock: Entrance and exit

In

Outline of IRID

1. Name International Research Institute for Nuclear Decommissioning (IRID) https://irid.or.jp/en/ R&D 2. Date of Establishment August 1, 2013 IRID 3. Membership (19 organizations) 2 research institutes International HRD JAEA etc. **Entities** 4 manufacturers Toshiba ESS, Hitachi-GE, MHI etc. 12 electric utilities, etc. TEPCO Holdings etc.

Introduction: R&D projects conducted by IRID



Contents

1. Introduction

- 2. Development of Remotely Operation Equipment for Dose Reduction
- 3. Development of Repairing Technology for PCV
- 4. Development of Investigation Technology for inside PCV
- 5. Development of Technology for Fuel Debris Retrieval
- 6. Nuclear Safety Enhancement

PCV : primary containment vessel

10 **Remote Decontamination Technology**

Needs for technological development

Humans cannot access the R/B because radiation levels are high in the R/B. It is necessary to improve work environments (dose reduction).

Reactor building (R/B)

Decontamination of work and moving

areas

Spent

fuel pool

PCV

For low places (floors and lower part of walls)

For upper floors





Remote Decontamination Technology

Upper wall of elevator

Site application (Unit 3)

Suction and dry ice blast decontamination was performed on 1st floor of the Unit 3 R/B from January 2016 to February 2016.

[]: Suction **[]**: Dry ice blast



Photos of transporting decontamination equipment from a container



Photo of decontamination equipment moving to the Unit 3 R/B



Entry route

Cable tray

West side wall

Contents

1. Introduction

- 2. Development of Remotely Operation Equipment for Dose Reduction
- 3. Development of Repairing Technology for PCV
- 4. Development of Investigation Technology for inside PCV
- 5. Development of Technology for Fuel Debris Retrieval
- 6. Nuclear Safety Enhancement

PCV : primary containment vessel



Repair Technology to Stop Water Leaks from Primary Containment Vessel (PCV)



Confirmation of Procedures for Full-scale Test Facility (JAEA Naraha Center for Remote Control Technology Development)

Purpose

 Creating of procedure manual by using a full-scale test facility considering actual onsite work to determine the applicability of the actual equipment.

Major work

- The procedures of the following three methods for water stops are tested to verify workability and performance of concrete placing.
 - Stop water leaks from vent pipes
 - Stop water leaks in the S/C by injecting filling
 - ③ Strengthen the S/C support column

Test period

Nov. 2016 – March 2018



Appearance of test facility



Verification test of workability to strengthen the S/C support column



Inside of the test facility (in the S/C))



Contents

- 1. Introduction
- 2. Development of Remotely Operation Equipment for Dose Reduction
- 3. Development of Repairing Technology for PCV
- Development of Investigation Technology for inside PCV
- 5. Development of Technology for Fuel Debris Retrieval
- 6. Nuclear Safety Enhancement

PCV : primary containment vessel

Muon Transmission Measurement

- Muons are secondary cosmic rays, which generate when radiation from space collides with the atmosphere of the Earth. The cosmic ray muons are high-energy particles and can pass through materials.
- Muon tomography can measure the number of muons that pass through the reactor building to image the density of materials such as X-ray. It can be used to image the distribution of fuel debris in the reactor pressure vessel (RPV). (Smaller number of muons will pass through high density regions so higher density regions show dark shadow).



16



©International Research Institute for Nuclear Decommissioning

Measurement Result of the Muon Transmission Method for Unit 3



17

Quoted from report released from the TEPCO Holding Inc. website.

IRID

Robot Investigation of the PCV interiors

Investigation of outside the pedestal (Unit 1) Investigation of inside the pedestal (Unit 2) ○ Remote-operated crawler robot for investigation Crawler **During investigation** dditional ahtina Winch Back camera a lighting Camera for travelling through guide pipe During Crawler Type I (when travelling investigation through guide pipe) 0 Front camera and lighting Camera for travelling CRD rail **OSuspension type** investigation equipment (A2' investigation) Winch confirmation camera (inside) Investigation of inside the pedestal (Unit 3) Sensor unit measuring camera Thruster for up-and-down U-shape type (when travelling Front camera on the floor) Taper Thruster for Radiation driving dosimetry Liaht LED **OSubmersible Crawling Robot** Φ20 mr Measuring camera





Investigation of the Unit 2 upper pedestal interior

(A2 investigation: January – February 2017)



Investigation of the Unit 2 upper pedestal interior

(A2 investigation: January – February 2017)

Upper pedestal interior (after image processing)



Investigation of the lower pedestal interior

(A2' investigation in January 2018)

Purpose of investigation

Confirmation of the state below the platform

Investigation procedure

(1)Inserting a guide pipe \Rightarrow (2)Extending a pipe \Rightarrow (3)Suspending a pan-tilt camera \Rightarrow (4)Investigation



IRID

©International Research Institute for Nuclear Decommissioning

Unit 2 investigation: Pedestal Floor



Bottom of the Unit 2 PCV (An overhead image)

Pedestal floor and wall Fuel debris? and a fuel assembly handle





Investigation of the Lower Pedestal Interior

(A2' investigation in February 2019)

TEPCO



Investigation of the Unit 2 Lower Pedestal Interior (A2' investigation in February 2019) Video: Unit #2 deposition at the lower pedestal area





Submersible Remote Operated Vehicle (ROV) (mockup vehicle)

Items Specifications Outer size Outer diameter : \$125mm Overall length : Approx.300m Weight Approx.2000g (in air) Radiation Radiation 200Gy				
Items Specifications Outer size Outer diameter : φ125mm Overall length : Approx.300m Weight Approx.2000g (in air) Radiation Radiation 200Gy	Thruster for up-and-down	NOR	Neut	ral buoyancy cable
Outer size Outer diameter : @125mm Overall length : Approx.300m Weight Approx.2000g (in air) Radiation Radiation 200Gy	A CONTRACTOR OF A CONTRACTOR OFTA CONT		Items	Specifications
Weight Approx. 2000g (in air) Radiation resistance 200Gy			Outer size	Outer diameter : φ125mm Overall length : Approx.300mm
Radiation resistance 200Gy	do,		Weight	Approx. 2000g (in air)
	123		Radiation resistance	200Gy
Erept comerce Light Back camera	Front compres	Back camera	Light	

IRID

Results of investigation for the Unit 3 PCV



Boat Type Access Equipment

Boat type access equipment which can move on a wide range of the water surface in the primary containment vessel (PCV) was developed.



Example: Guide ring installation

- Diameter: φ25cm
- Length: Approx. 1.1m
- Thrust: Over 25N

Appearance of the boat type access equipment



Travelling line of the equipment



Boat Type Access Equipment (Video)





Boat Type Access Equipment (Investigation inside PCV through X-2 penetration)

Six kinds of boat type access and investigation equipment with submersible functions will be prepared for each function.



*Thickness and existence of deposition and fuel debris are uncertain. The above figures are image descriptions.



Report of the 107th study committee of specific nuclear facility monitoring and evaluation on April 14, 2023.

[Reference] Panoramic photo images taken from the pedestal opening



-

Report of the 107th study committee of specific nuclear facility monitoring and evaluation on April 14, 2023.

[Reference] Concrete remains of the right pedestal opening (1/2)

TEPCO

- Tokyo Electric Power Company (TEPCO) Holdings, Inc. investigated the Unit 1 pedestal to confirm the concrete that likely remain outside the pedestal (Bolts that were installed before the accident were confirmed to be fixed). The investigation on March 2023 confirmed the inside of the pedestal wall.
- TEPCO assumes that the lost concrete of the pedestal outer wall opening in right side would be limited.
- The investigation found that reinforcing steels of the outside pedestal have remained 7 pieces in the right opening part and 11 pieces in the left side. The earthquake resistance should be evaluated based on 64° that is equivalent to the angle in accordnce with the opening angle.





©Tokyo Electric Power Company Holdings, Inc. All Rights Reserved

Photo 2: Concrete remains can be seen from the pedestal opening

Report of the 107th study committee of specific nuclear facility monitoring and evaluation on April 14, 2023.

ΤΞΡϹΟ

[Reference] Concrete remains of the right pedestal opening (2/2)



Photos processed by TEPCO Holdings Inc. 33

Arm Type Access Equipment

- The arm type access equipment was manufactured which can access on a wide range through the PCV penetration for maintenance of control rod drive mechanism.
 - Total length of the arm: Approx. 22m
 - Investigation equipment up to 10kg can be loaded.



Arm type access equipment



Configuration of Access Equipment





Arm Type Access Equipment (video)







Contents

- 1. Introduction
- 2. Development of Remotely Operation Equipment for Dose Reduction
- 3. Development of Repairing Technology for PCV
- 4. Development of Investigation Technology for inside PCV
- 5. Development of Technology for Fuel Debris Retrieval

6. Nuclear Safety Enhancement

PCV : primary containment vessel

Development of the Large Scale Fuel Debris Retrieval Method

To increase the retrieval scale, large-scale apparatus was installed.

> Fuel debris retrieval methods are being developed:

Three side-access retrieval methods and two top-access methods.

- The retrieval amount of fuel debris will be gradually increased to up to 300 kg per day.
- > NDF assessed that Unit 3 is an appropriate unit to start fuel

debris retrieval (by side-access method).

The obtained technology, know-how, organization structure, experiences Carry-out and removal of the whole structure and lesson-learnt will be expected to be utilized for design, procurement, construction and operation.
A concept of separator remove
A concept of jet pump





Fuel Debris Retrieval Technology



Unauthorized copy and reproduction prohibited. ©International Research Institute for Nuclear Decommissioning

Access Tunnel Method

- The access tunnel method is required to connect a heavy-lift tunnel (approximately 800 ton) with the primary containment vessel (PCV) from outside the reactor building through the precise position control system.
- Delivery technology for curved heavy-lift tunnel in narrow spaces has been developed with applied heavy delivery technology experienced in bridge constructions.



Element test of tunnel building technology



[Example of the top-access method]: Methods for Removing and Transporting the Entire Structures





Technology for Containing, Transferring and Storing Fuel Debris

Design of canister ⇒Responding to issues specific to the Fukushima Daiichi

- High-burnup and the enrichment \rightarrow High reactivity
- Molten products mixed with concrete

 Hydrogen generation caused by radiolysis of moisture containing concrete
- Molten products with sea water injected and instrumental cables, etc. \rightarrow Impact of salt and contamination of impurities

Transport method (Ex. Partial submersion side-access method



Decommissioning

Contents

- 1. Introduction
- 2. Development of Remotely Operation Equipment for Dose Reduction
- 3. Development of Repairing Technology for PCV
- 4. Development of Investigation Technology for inside PCV
- 5. Development of Technology for Fuel Debris Retrieval
- 6. Nuclear Safety Enhancement

PCV : primary containment vessel

Ensuring the Safety When Retrieving Fuel Debris

Risks necessary to be considered

- 1. Cooling
 - •••Decay heat has decreased over a long time after the accident, however it must be cooled for a certain period of time. There is a risk of losing its functions.
- 1. Confinement
 - •••There is a risk of releasing dust to be generated when cutting and chipping debris.
- 2. Fire and explosion (inactivation)
 - •••There is a risk of fire and hydrogen explosion when cutting and chipping debris.
- 3. Criticality

•••There is a risk of criticality caused by changing the shape of debris during fuel debris retrieval.

Ensuring the Safety When Retrieving Fuel Debris



Decommissioning

Concepts of Study: Procedures and Feasibility of System Design

- On the basis of the safety requirement setting (tentative), functional requirements from the throughput will be added.
- The safety requirements should be basically unchangeable, however potential current risks an work risks (i.e. work dose of radioactivity) may be reviewed depending on estimated work and evaluation.



Concepts of Defense in Depth (proposed) (1/2)

- Regardless five levels of defense in depth for the light water reactor, an additional defense level was newly determined for fuel debris retrieval.
- Specifically, three defense levels, "Prevention of abnormal operation," "Control of abnormal operation and termination of failures" and "Impact mitigation of accidents of the public" were determined and organized depending on the safety functions.



Concepts of Defense in Depth (proposed) (2/2)

The decommissioning of the Fukushima Daiichi differs from the one of normal plants. Considering a special circumstance of the Fukushima Daiichi that the decommissioning of the accident plant (work in high radiation environments) will maintain for a long period of time and has a potential hazard of the final level. The following items were considered to establish the defense in depth.

[Exposure of workers]

The effects of worker's exposure associated with installation work should be also considered in addition to effects of the exposure during fuel debris retrieval and exposure reduction of the public. The purpose of exposure reduction is to totally reduce exposure for the public and workers.

[Event progression]

At the time of functional loss, existing facilities will respond to events that rapidly progress. Transportable equipment will also be utilized for slow-progression-events.

[Robustness of existing facilities]

The robustness of the facility is required for events that rapidly progress and its function is expected to be used. In case that the function is expected to be used for only normal time and is not significant even though it takes time to restore the function at the time of functional loss, the robustness will not be required.

Summary

- In the development of debris retrieval work technology, detailed on-site surveys are important for ensuring safety and earlier decommissioning work.
- Preparations for detailed investigation in the containment vessel and the taking out of small amounts of debris are underway.
- Risk assessment and defense in depth have been examined, and the conceptual design of debris retrieval systems has been advanced.
- Technology development concerning nuclear safety such as criticality control and hydrogen generation control are also being advanced.



End of presentation